

Compton Suppressed Gamma Spectroscopy of Spent Fuel Inventories

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Background



- A little about me:
 - Texas A&M University-Kingsville (Junior Fall 2016)
 - Mechanical Engineering major, and Nuclear Engineering minor
- Project Origins:
 - Idaho National Laboratory (ATR)
 - Penn State University

Motivation of Project:

- To successfully determine the burnup of highly enriched uranium (HEU) fuel in the Neutron Beam Split-core Reactor (NBSR) at the NCNR.
- Benchmark the power and burnup of the NBSR.

INL Study (2013)

- A Feasibility and Optimization Study to Determine Cooling Time and Burnup of Advanced Test Reactor Fuels Using a Nondestructive Technique by Jorge Navarro
- Determined <u>burnup and cooling time</u> of fuel using isotopic γ ray peak area, and <u>peak area ratios</u>.
- Relationship was found between burnup and isotopes including ¹³⁷Cs, and the ratios of other isotopes correlated with the cooling time (such as ¹⁴⁴Ce/ ¹³⁷Cs).
- Isotope ratios were correlated to <u>cancel</u> <u>out the geometry</u> of the measurement.



Penn State Study (2014)

Study of Compton Suppression for Use in Spent Nuclear Fuel Assay by Sarah Bender

- A fuel sample from the Penn State Breazeale Reactor was measured using <u>Compton Suppression</u>.
- Used concrete collimator built into the fuel pool to study aged LEU fuel.
- Eight additional photopeaks were unmasked, allowing for <u>identification of</u> <u>more isotopes</u> in the fuel sample.



Penn State Study (2014)



Compton Suppression

Compton Scattering



How do we apply Compton Suppression?





NBSR Fuel

The Reactor:

- D₂O Coolant, moderator, and reflector
- 30 fuel elements with 38.5 day fuel cycles
- ~20 MW Thermal Power

The Fuel:

- ▶ 93% U₃O₈ + Al
- 17 fuel plates per region
- 2 regions per fuel element

Burnup: If 10% of an initial isotope underwent fission, the burnup is 10%. In general: $\frac{MW \cdot d}{Metric Tonne} = \frac{Power \times Time Spent}{Initial Mass of Fuel}$

Initial Mass of Fuel



Challenges

We wanted to measure spectra from different spent fuel elements using a HPGe detector and a BGO detector.

- \blacktriangleright We had to channel a γ beam from the fuel elements to the detectors.
- > The detectors had to be setup in a way that accommodates Compton suppression.
- > It had to be safe: health physics, crane lift, and γ beam control.
- Not a single thing besides the collimator and its peripherals was to touch the water.
- Had to be easy to assemble and disassemble for multiple uses for different γ beam sizes.
- Mechanical issues to be addressed: structure, buoyancy, material selection, manufacturing
- Cost Effectiveness
- Detectors Setup and Calibration
- EVERYTHING MUST WORK!!!!

Results?



Radiation Safety



Radiation Safety

Attenuation Coefficient (µ):

$$D = D_0 e^{-\mu x}$$



Tenth Value Layer (TVL):

 $TVL = \frac{ln(10)}{\mu}$

Calculated Results:

		μ (1/ft)	TVL (ft)
	1048	1.8697	1.23
	1049	1.6869	1.36
	1050	1.8735	1.23
	1051	1.8437	1.25
\langle	AVG	1.81845	1.27





Where do we put the detectors?



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Where do we put the detectors?



The Apparatus



The Collimator

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Structural Analysis

There is a risk of dropping detectors in the pool.

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- Treat system as a structural beam, and find the center of mass for the platform.
- Center of mass was within the safety zone.



Buoyancy Analysis

In order for the collimator to work properly, it must sink in.

Original thought: 2"OD 1.65"ID tubes for collimator

Health Physics Concerns \rightarrow make Collimator smaller (1"OD, 0.93"ID)

 $F_{B,applied} = \rho_{H_2O} \times g \times V_{df,max} - w_{Col} \approx 5 \ lb$

5 lb deficit

But since it's held in place, would it REALLY bend too much?



Detector Calibration

- Done using GENIE-2000
- Energy Calibration using Co-60 and Eu-154 Sources









Initial Results



Initial Results





Peak Fitting



Peaks were refitted in Hypermet-PC
 Δ¹³⁴Cs/Δ¹³⁷Cs ratio was calculated with the refitted peaks

Pictures Courtesy of Danyal Turkoglu



Isotope Buildup over fuel life (MCNP)



0 Relative Concentration 7 E-3 9 6+3 0 6+3 0 0+1 0 0+1 0

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Burnup of fuel (MCNP depletion study)

8000 7000 6000 (SVD4 2000 4000 3000 2000 1000 0 0.01 0.02 0.03 0.05 0.07 0.09 0.04 0.06 0.08 0.1 0 RATIO (134Cs/137Cs)

Burnup vs Cs Ratio

BURNUP!!

$$\frac{\Delta^{134}\text{Cs}}{\Delta^{137}\text{Cs}} = \frac{\varepsilon_{134}}{\varepsilon_{137}} \times \frac{CR_{134}}{CR_{137}} \times \frac{\lambda_{137}I_{662}}{\lambda_{134}I_{795}} \times \frac{e^{\lambda_{134}t_{cool}}}{e^{\lambda_{137}t_{cool}}}$$
Experiment Efficiency Count Rate Specific γ emission rate Cooling Time

Fuel Element	Isotope Ratio	Burnup (MW-day)	Power* (MW)
S1025 (July 2015)	0.0751	5329	19.8
S1036 (Jan 2016)	0.0705	5100	18.9

*38.5 d cycles assumed (7-cycle fuel)

Calculated error due to background corrected gamma peaks area is +/-0.04 MW

Burnup Results



Burnup vs Ratio

Conclusion

- Assembled the entire experiment and proved that it works
- Addressed health physics concerns and completed safety evaluation
- Observed several isotopes from fuel, and was able to determine burnup
- This is the first time this is ever been done

Future Work

- Make a collimator that works better
- Improve stability of apparatus
- Longer count times (overnight)
- Measuring additional elements
- Evolve this experiment into a gamma spectroscopy scanning apparatus

Acknowledgements

Thomas Newton Dan Hughes Daniel Flynn James Moody Thomas Johnston Samuel MacDavid William Clow Julie Borchers Paul Brand Attila Halacsy Robert William Gregory Heller David Brown Doug Johnson Don Lopez Joe Dura

And all the SURF directors and the SURF Program

³² Acknowledgements

A special thank you to these individuals:

Daniel Mattes



Danyal Turkoglu



He's a Mechanical Engineer. It automatically makes him awesome

Timothy Barvitskie

Guided me through μ and TVL calculations (and helped out with Genie...and helped out with other details of the project)



Taught me almost everything I know about detectors





Acknowledgments

My Mentor:



Bryan Eyers





